www.regualtions.gov and search for Docket No. NRC–2014–0079. Mail comments to the Acting NRC Clearance Officer, Fajr Majeed (T–5 F50), U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001.

Questions about the information collection requirements may be directed to the Acting NRC Clearance Officer, Fajr Majeed (T–5 F50), U.S. Nuclear Regulatory Commission Washington, DC 20555–0001; telephone: 301–415–6736, or by email to

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Dated at Rockville, Maryland, this 15th day of May, 2014.

For the Nuclear Regulatory Commission.

Fajr Majeed,

Acting NRC Clearance Officer, Office of Information Services.

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NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-266 and 50-301; NRC-2014-0117]

Exemption for NextEra Energy Point Beach, LLC; Point Beach Nuclear Plant, Units 1 and 2

AGENCY: Nuclear Regulatory Commission.

ACTION: Exemption; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is issuing an exemption in response to a June 4, 2013, request from NextEra Energy Point Beach, LLC, requesting an exemption to use of a different fuel rod cladding material (Optimized ZIRLOTM).

ADDRESSES: Please refer to Docket ID NRC–2014–0117 when contacting the NRC about the availability of information regarding this document. You may access publicly-available information related to this document using any of the following methods:

- Federal Rulemaking Web site: Go to http://www.regulations.gov and search for Docket ID NRC-2014-0117. Address questions about NRC dockets to Carol Gallagher; telephone: 301-287-3422; email: Carol.Gallagher@nrc.gov. For technical questions, contact the individual listed in the FOR FURTHER INFORMATION CONTACT section of this document.
- NRC's Agencywide Documents Access and Management System (ADAMS): You may access publicly available documents online in the NRC Library at http://www.nrc.gov/readingrm/adams.html. To begin the search, select "ADAMS Public Documents" and

then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to pdr.resource@nrc.gov. The ADAMS accession number for each document referenced in this document (if that document is available in ADAMS) is provided the first time that a document is referenced.

• NRC's PDR: You may examine and purchase copies of public documents at the NRC's PDR, Room O1–F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

FOR FURTHER INFORMATION CONTACT:

Terry A. Beltz, Office of Nuclear Reactor Regulation, telephone: 301–415–3049; email: *Terry.Beltz@nrc.gov*, U.S. Nuclear Regulatory Commission, Washington DC 20555–0001.

I. Background

NextEra Energy Point Beach, LLC (NextEra or the licensee) is the holder of renewed Facility Operating License Nos. DPR-24 and DPR-27, which authorize operation of the Point Beach Nuclear Plant (Point Beach), Units 1 and 2, respectively. The license provides, among other things, that the facility is subject to all rules, regulations, and orders of the NRC now or hereafter in effect.

The facility consists of two pressurized-water reactors located in Manitowac County in Wisconsin.

II. Request/Action

Pursuant to Section 50.12 of Title 10 of the Code of Federal Regulations (10 CFR), "Specific exemptions," the licensee has, by letter dated June 4, 2013 (ADAMS Accession No. ML13155A239), requested an exemption from 10 CFR 50.46, "Acceptance criteria for emergency core cooling systems [ECCS] for light-water nuclear power reactors,' and 10 CFR Part 50, Appendix K, "ECCS Evaluation Models," to allow the use of fuel rod cladding with Optimized ZIRLOTM alloy for future reload applications. The regulations in 10 CFR 50.46 contain acceptance criteria for the ECCS for reactors fueled with zircaloy or ZIRLO $^{\mathrm{TM}}$ fuel rod cladding material. In addition, paragraph I.A.5 of Appendix K to 10 CFR Part 50 requires that the Baker-Just equation be used to predict the rates of energy release, hydrogen concentration, and cladding oxidation from the metal/water reaction. The Baker-Just equation assumes the use of a zirconium alloy, which is a material different from Optimized ZIRLO $^{\rm TM}.$ Thus, the strict application of these regulations does not permit the use of fuel rod cladding material other than

zircaloy or ZIRLOTM. Because the material specifications of Optimized ZIRLOTM differ from the specifications for zircaloy or ZIRLOTM, and the regulations specify a cladding material other than Optimized ZIRLOTM, a plant-specific exemption is required to allow the use of, and application of these regulations to, Optimized ZIRLOTM at Point Beach.

The exemption request relates solely to the cladding material specified in these regulations (i.e., fuel rods with zircaloy or ZIRLOTM cladding material). This exemption would allow application of the acceptance criteria of 10 CFR 50.46 and 10 CFR Part 50, Appendix K, to fuel assembly designs using Optimized ZIRLOTM fuel rod cladding material. In its letter dated June 4, 2013, the licensee indicated that it was not seeking an exemption from the acceptance and analytical criteria of these regulations. The intent of the request is to allow the use of the criteria set forth in these regulations for the use of Optimized ZIRL $\breve{\mathsf{O}}^{\mathsf{TM}}$ fuel rod cladding material at Point Beach.

III. Discussion

Pursuant to 10 CFR 50.12, the Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of 10 CFR Part 50 when: (1) The exemptions are authorized by law, will not present an undue risk to public health or safety, and are consistent with the common defense and security; and (2) when special circumstances are present. Under 10 CFR 50.12(a)(2)(ii), special circumstances include, among other things, when application of the specific regulation in the particular circumstance would not serve, or is not necessary to achieve, the underlying purpose of the rule.

A. Special Circumstances

Special circumstances, in accordance with 10 CFR 50.12(a)(2)(ii), are present whenever application of the regulation in the particular circumstances is not necessary to achieve the underlying purpose of the rule. The underlying purpose of 10 CFR 50.46 and Appendix K to 10 CFR Part 50 is to establish acceptance criteria for ECCS performance to provide reasonable assurance of safety in the event of a lossof-coolant accident (LOCA). Although the regulations in 10 CFR 50.46 and Appendix K are not expressly applicable to Optimized ZIRLOTM, the evaluations described in the following sections of this exemption show that the purpose of the regulations are met by this exemption in that, subject to certain conditions, the acceptance criteria are valid for Optimized ZIRLOTM fuel cladding material, Optimized ZIRLOTM would maintain better post-quench ductility, and the Baker-Just correlation conservatively bounds LOCA scenario metal-water reaction rates and is applicable to Optimized ZIRLOTM. Thus, a strict application of the rule (which would preclude the applicability of ECCS performance acceptance criteria to, and the use of, Optimized ZIRLO $^{\mathrm{TM}}$ fuel cladding material) is not necessary to achieve the underlying purposes of 10 CFR 50.46 and Appendix K to 10 CFR Part 50. The purpose of these regulations is achieved through application of the requirements to the use of Optimized ZIRLOTM fuel rod cladding material. Therefore, the special circumstances required by 10 CFR 50.12(a)(2)(ii) for the granting of an exemption exist.

B. Authorized by Law

This exemption would allow the use of Optimized ZIRLO TM fuel rod cladding material for future reload operations at Point Beach. As stated above, 10 CFR 50.12 allows the NRC to grant exemptions from the requirements of 10 CFR Part 50 provided that special circumstances are present. As described above, the NRC staff has determined that special circumstances exist to grant the requested exemption. In addition, granting the exemption will not result in a violation of the Atomic Energy Act of 1954, as amended, or the Commission's regulations. Therefore, the exemption is authorized by law.

C. No Undue Risk to Public Health and Safety

Section 10 CFR 50.46 requires that each boiling or pressurized light-water nuclear power reactor fueled with uranium dioxide pellets within cylindrical zircalov or ZIRLOTM cladding must be provided with an ECCS that must be designed so that its calculated cooling performance following a postulated loss-of-coolant accident (LOCA) conforms to the criteria set forth in paragraph (b) of this section. The underlying purpose of 10 CFR 50.46 is to establish acceptance criteria for adequate ECCS performance. As previously documented in the NRC staff's safety evaluation dated June 10, 2005 (ADAMS Accession No. ML051670395), of topical reports submitted by Westinghouse Electric Company, LLC (Westinghouse), and subject to compliance with the specific conditions of approval established therein, the NRC staff found that Westinghouse demonstrated the applicability of these ECCS acceptance

criteria to Optimized ZIRLOTM. Ring compression tests performed by Westinghouse on Optimized ZIRLOTM (see WCAP-14342-A & CENPD-404-NP-A at ADAMS Accession No. ML062080569) demonstrate an acceptable retention of post-quench ductility up to 10 CFR 50.46 limits of 2,200 degrees Fahrenheit and 17 percent equivalent clad reacted. Furthermore, the NRC staff has concluded that oxidation measurements provided by the licensee in letter LTR-NRC-07-58 from Westinghouse to the NRC, "SER Compliance with WCAP-12610-P-A & CENPD-404-P-A, Addendum 1-A, 'Optimized ZIRLOTM,' " dated November 6, 2007 (public version located at ADAMS Accession No. ML073130560), illustrate that oxide thickness and associated hydrogen pickup for Optimized ZIRLOTM at any given burnup would be less than both zircaloy-4 and ZIRLOTM. Hence, the NRC staff concludes that Optimized ZIRLOTM would be expected to maintain better post-quench ductility than ZIRLOTM. This finding is further supported by an ongoing LOCA research program at Argonne National Laboratory, which has identified a strong correlation between cladding hydrogen content (caused by in-service corrosion) and post-quench ductility

In addition, the provisions of 10 ČFR 50.46 require the licensee to periodically evaluate the performance of the ECCS, using currently approved LOCA models and methods, to ensure that the fuel rods will continue to satisfy 10 CFR 50.46 acceptance criteria. In its letter dated June 4, 2013, the licensee stated that it will evaluate fuel assemblies using Optimized ZIRLOTM fuel rod cladding material using NRCapproved analytical methods and plantspecific models to address the changes in cladding material properties. The NRC staff concludes that granting the exemption to allow the licensee to use Optimized ZIRLOTM fuel rod cladding material and apply 10 CFR 50.46 criteria would not diminish this requirement of periodic evaluation of ECCS performance. Thus, the underlying purpose of the rule to maintain postquench ductility in the fuel cladding material through ECCS performance criteria will continue to be achieved for Point Beach.

Paragraph I.A.5 of Appendix K to 10 CFR Part 50 states that the rates of energy release, hydrogen concentration, and cladding oxidation from the metalwater reaction shall be calculated using the Baker-Just equation. Since the Baker-Just equation presumes the use of zircaloy clad fuel, strict application of this provision of the rule would not

permit use of the equation for Optimized ZIRLOTM fuel rod cladding material for determining acceptable fuel performance. The underlying purpose of this regulation, however, is to ensure that analyses of fuel response to LOCAs are conservatively calculated. In its evaluation of the approved topical reports, the NRC staff previously found that metal-water reaction tests performed by Westinghouse on Optimized ZIRLOTM (see Appendix B of WCAP-12610-P-A and CENPD-404-P-A, Addendum 1-A) demonstrate conservative reaction rates relative to the Baker-Just equation, and that the Baker-Just equation conservatively bounds post-LOCA scenarios of, and applicable to, Optimized ZIRLO™ fuel rod cladding. Thus, the NRC staff determined that the strict application of Appendix K, Paragraph I.A.5 (which would preclude its applicability to, and the use of, Optimized ZIRLOTM) is not necessary to achieve the underlying purpose of the rule in these circumstances. Since these evaluations demonstrate that the underlying purpose of the rule will be met, there will be no undue risk to the public health and safety.

D. Consistent With the Common Defense and Security

The licensee's exemption request is to allow the application of an improved fuel rod cladding material to the regulations in 10 CFR 50.46 and paragraph I.A.5 of Appendix K to 10 CFR 50. In its letter dated June 4, 2013, the licensee stated that all the requirements and acceptance criteria will be maintained. The licensee is required to handle and control special nuclear material in these assemblies in accordance with its approved procedures. This change to reactor core internals is adequately controlled by NRC requirements and is not related to security issues. Therefore, the NRC staff determined that this exemption does not impact, and thus is consistent with, the common defense and security.

E. Environmental Considerations

The NRC staff determined that the exemption discussed herein meets the eligibility criteria for the categorical exclusion set forth in 10 CFR 51.22(c)(9) because it is related to a requirement concerning the installation or use of a facility component located within the restricted area, as defined in 10 CFR Part 20, and issuance of this exemption involves: (i) No significant hazards consideration, (ii) no significant change in the types or a significant increase in the amounts of any effluents that may be released offsite, and (iii) no significant

increase in individual or cumulative occupational radiation exposure. Therefore, in accordance with 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the NRC's consideration of this exemption request. The basis for the NRC staff's determination is discussed as follows with an evaluation against each of the requirements in 10 CFR 51.22(c)(9)(i)—(iii).

Requirements in 10 CFR 51.22(c)(9)(i)

The NRC staff evaluated whether the exemption involves no significant hazards consideration using the standards described in 10 CFR 50.92(c), as presented below:

1. Does the proposed exemption involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

The proposed exemption would allow the use of Optimized ZIRLOTM fuel rod cladding material in the reactors. The NRC approved topical report WCAP-12610-P-A and CENPD-404-P-A, Addendum 1-A "Optimized ZIRLOTM, " prepared by Westinghouse, addresses Optimized ZIRLOTM and demonstrates that Optimized ZIRLOTM has essentially the same properties as currently licensed ZIRLO®. The fuel cladding itself is not an accident initiator and does not affect accident probability. Use of Optimized ZIRLOTM fuel cladding material will continue to meet all 10 CFR 50.46 acceptance criteria and, therefore, will not increase the consequences of an accident.

Therefore, the proposed exemption does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed exemption create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The use of Optimized ZIRLOTM fuel rod cladding material will not result in changes in the operation or configuration of the facility. Topical Reports WCAP-12610-P-A and CENPD-404-P-A demonstrated that the material properties of Optimized ZIRLOTM are similar to those of standard ZIRLO®. Therefore, Optimized ZIRLOTM fuel rod cladding material will perform similarly to those fabricated from standard ZİRLO®, thus precluding the possibility of the fuel cladding becoming an accident initiator and causing a new or different type of accident. Therefore, the proposed exemption does not create the

possibility of a new or different kind of accident from any previously evaluated.

3. Does the proposed exemption involve a significant reduction in a margin of safety?

Response: No.

The proposed exemption will not involve a significant reduction in the margin of safety because it has been demonstrated that the material properties of the Optimized ZIRLOTM are not significantly different from those of standard ZIRLO®. Optimized ZIRLOTM is expected to perform similarly to standard ZIRLO® for all normal operating and accident scenarios, including both LOCA and non-LOCA scenarios. For LOCA scenarios, where the slight difference in Optimized ZIRLOTM material properties relative to standard ZIRLO® could have some impact on the overall accident scenario, plant-specific LOCA analyses using Optimized ZIRLOTM properties will demonstrate that the acceptance criteria of 10 CFR 50.46 have been satisfied.

Therefore, the proposed exemption does not involve a significant reduction in a margin of safety.

Based on the above evaluation of the standards set forth in 10 CFR 50.92(c), the NRC staff concludes that the proposed exemption involves no significant hazards consideration. Accordingly, the requirements of 10 CFR 51.22(c)(9)(i) are met.

Requirements in 10 CFR 51.22(c)(9)(ii)

The proposed exemption would allow the use of Optimized ZIRLOTM fuel rod cladding material in the reactors. Optimized ZIRLOTM has essentially the same material properties and performance characteristics as the currently licensed ZIRLO[®] cladding. Thus, the use of Optimized ZIRLOTM fuel rod cladding material will not significantly change the types of effluents that may be released offsite, or significantly increase the amount of effluents that may be released offsite. Therefore, the requirements of 10 CFR 51.22(c)(9)(ii) are met.

Requirements in 10 CFR 51.22(c)(9)(iii)

The proposed exemption would allow the use of Optimized ZIRLOTM fuel rod cladding material in the reactors. Optimized ZIRLOTM has essentially the same material properties and performance characteristics as the currently licensed ZIRLO® cladding. Thus, the use of Optimized ZIRLOTM fuel rod cladding material will not significantly increase individual occupational radiation exposure, or significantly increase cumulative occupational radiation exposure.

Therefore, the requirements of 10 CFR 51.22(c)(9)(iii) are met.

Conclusion

Based on the above, the NRC staff concludes that the proposed exemption meets the eligibility criteria for the categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, in accordance with 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the NRC's proposed issuance of this exemption.

IV. Conclusions

Accordingly, the Commission has determined that, pursuant to 10 CFR 50.12, the exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security. Also, special circumstances pursuant to 10 CFR 50.12(a)(2)(ii) are present. Therefore, the Commission hereby grants NextEra an exemption from the requirements of 10 CFR 50.46 and Appendix K to 10 CFR Part 50, to allow the application of those criteria to, and the use of, Optimized ZIRLOTM fuel rod cladding material at the Point Beach Nuclear Plant, Units 1 and 2.

This exemption is effective upon issuance.

Dated at Rockville, Maryland, this 9th day of May 2014.

For The Nuclear Regulatory Commission.

Michele G. Evans,

Director, Division of Operating Reactor Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 2014–11615 Filed 5–19–14; 8:45 am] BILLING CODE 7590–01–P

NUCLEAR REGULATORY COMMISSION

[IA-13-059; NRC-2014-0115]

In the Matter of Richard Brian Smith

AGENCY: Nuclear Regulatory Commission.

ACTION: Order; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is issuing an order prohibiting Richard Brian Smith's involvement in any NRC-licensed activities for a period of five years. This Order is based on Richard Brian Smith having twice tested positive for an illegal substance during random fitnessfor-duty (FFD) tests while holding an NRC operator's license at the Grand Gulf Nuclear Station and the results of an NRC investigation.

DATES: Effective Date: See attachment.