

Committee will discuss the recommendations of the Planning and Procedures Subcommittee regarding items proposed for consideration by the full Committee during future meetings. Also, it will hear a report of the Planning and Procedures Subcommittee on matters related to the conduct of ACRS business, and organizational and personnel matters relating to the ACRS.

**2:15 p.m.–3:15 p.m.: Report Regarding Recent Operating Events (Open)**—Report by the cognizant ACRS member regarding recent operating events of interest.

**3:15 p.m.–7 p.m.: Proposed ACRS Reports (Open/Closed)**—The Committee will discuss proposed ACRS reports.

#### **Saturday, October 12, 2002**

**8:30 a.m.–1 p.m.: Proposed ACRS Reports (Open)**—The Committee will discuss proposed ACRS reports.

**1 p.m.–1:30 p.m.: Miscellaneous (Open)**—The Committee will discuss matters related to the conduct of Committee activities and matters and specific issues that were not completed during previous meetings, as time and availability of information permit. The Committee will also discuss its plans for preparing a “white paper” on the use of PRA in the regulatory decisionmaking process.

Procedures for the conduct of and participation in ACRS meetings were published in the **Federal Register** on October 3, 2001 (66 FR 50462). In accordance with those procedures, oral or written views may be presented by members of the public, including representatives of the nuclear industry. Electronic recordings will be permitted only during the open portions of the meeting. Persons desiring to make oral statements should notify the Associate Director for Technical Support named below five days before the meeting, if possible, so that appropriate arrangements can be made to allow necessary time during the meeting for such statements. Use of still, motion picture, and television cameras during the meeting may be limited to selected portions of the meeting as determined by the Chairman. Information regarding the time to be set aside for this purpose may be obtained by contacting the Associate Director prior to the meeting. In view of the possibility that the schedule for ACRS meetings may be adjusted by the Chairman as necessary to facilitate the conduct of the meeting, persons planning to attend should check with the Associate Director if such rescheduling would result in major inconvenience.

Further information regarding topics to be discussed, whether the meeting

has been canceled or rescheduled, the Chairman’s ruling on requests for the opportunity to present oral statements, and the time allotted therefor can be obtained by contacting Dr. Sher Bahadur, Associate Director for Technical Support (301–415–0138), between 7:30 a.m. and 4:15 p.m., EDT.

ACRS meeting agenda, meeting transcripts, and letter reports are available through the NRC Public Document Room at [pdrc@nrc.gov](mailto:pdrc@nrc.gov), or by calling the PDR at 1–800–397–4209, or from the Publicly Available Records System (PARS) component of NRC’s document system (ADAMS) which is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> or <http://www.nrc.gov/reading-rm/doc-collections/> (ACRS & ACNW Mtg schedules/agendas).

Videoteleconferencing service is available for observing open sessions of ACRS meetings. Those wishing to use this service for observing ACRS meetings should contact Mr. Theron Brown, ACRS Audio Visual Technician (301–415–8066), between 7:30 a.m. and 3:45 p.m., EDT, at least 10 days before the meeting to ensure the availability of this service. Individuals or organizations requesting this service will be responsible for telephone line charges and for providing the equipment and facilities that they use to establish the videoteleconferencing link. The availability of videoteleconferencing services is not guaranteed.

Dated: September 20, 2002.

**Annette Vietti-Cook,**

*Secretary of the Commission.*

[FR Doc. 02–24437 Filed 9–25–02; 8:45 am]

**BILLING CODE 7590–01–P**

### **NUCLEAR REGULATORY COMMISSION**

#### **Draft Decommissioning Questions and Answers Regarding Clarification of License Termination Guidance of the Nuclear Regulatory Commission’s Office Nuclear Material Safety and Safeguards; Notice of Availability**

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice of availability and request for public comment.

**SUMMARY:** The Nuclear Regulatory Commission’s (NRC) Office of Nuclear Material Safety and Safeguards (NMSS) is announcing the availability of draft decommissioning questions and answers regarding clarification of license termination guidance, for public comment.

The Nuclear Energy Institute (NEI) and NRC staff identified an approach to clarify existing guidance associated with the License Termination Rule (10 CFR part 20, subpart E), in concert with NMSS’ decommissioning guidance consolidation project. Under this approach, NEI’s License Termination Task Force (Task Force) generated questions (Qs) associated with decommissioning issues that are common to the industry. The Task Force also proposed answers (As) to the questions and submitted the Q&As to NRC staff for review. NRC staff reviewed the Q&As and the supporting technical bases and provided comments to NEI on September 28, 2001. An open meeting was held between NRC, NEI, and industry representatives on December 4, 2001, to discuss each Q&A and the technical issues to ensure that the questions were properly asked and answered and were supported by a defensible technical basis. NRC staff and NEI further developed the Q&As so that they adequately reflect NRC regulations and guidance and include a sound technical basis.

As a result of this cooperation, eight Q&As have been found acceptable by NRC staff. Seven of the Q&As were to be incorporated into the draft document “Consolidated NMSS Decommissioning Guidance: Characterization, Survey, and Determination of Radiological Criteria” (NUREG–1757, Volume 2) to solicit public comment on them. However, two Q&As were inadvertently omitted. Therefore, five Q&As are included in Volume 2 of NUREG–1757, and three Q&As are included in the “supplementary information” section of this notice. Volume 2 of NUREG–1757 is being published for public comment on or close to the date of this notice. NRC is seeking public comment on the Q&As and Volume 2 of NUREG–1757 in order to receive feedback from the widest range of interested parties and to ensure that all information relevant to developing the document is available to the NRC staff. These draft documents are being issued for comment only and are not intended for interim use. The NRC will review public comments received on the draft documents. Suggested changes will be incorporated, where appropriate, in response to those comments, and a final document will be issued for use. The final Q&As will be included in the text of the final document of Volume 2 of NUREG–1757.

**DATES:** Comments on this draft document should be submitted by December 26, 2002. Comments received after that date will be considered to the extent practicable.

**ADDRESSES:** Members of the public are invited and encouraged to submit written comments to: Duane W. Schmidt, Project Manager, Office of Nuclear Material Safety and Safeguards, Mail Stop T-7F27, U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001. Hand-deliver comments to: 11555 Rockville Pike, Rockville, MD, between 7:30 a.m. and 4:15 p.m., Federal workdays. Comments may also be sent electronically to *decomcomments@nrc.gov*. Copies of comments received may be examined at the ADAMS Electronic Reading Room on the NRC web site, and the NRC Public Document Room, 11555 Rockville Pike, Room O-1F21, Rockville, MD 20852. The NRC Public Document Room is open from 7:45 a.m. to 4:15 p.m., Monday through Friday, except on Federal holidays.

**FOR FURTHER INFORMATION, CONTACT:** Duane W. Schmidt, Mail Stop T-7F27, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001. Telephone: (301) 415-6919; Internet: *dws2@nrc.gov*.

#### SUPPLEMENTARY INFORMATION:

##### Question 5

What are acceptable methods to characterize embedded piping and buried piping?

##### Answer to Question 5

Several methods have been used to characterize the residual activity within embedded pipe, and these methods can be used for buried piping, as well. By definition, "embedded piping" is piping (e.g., part of a plant system) that is found in buildings and encased in concrete floors and walls, while "buried piping" is piping (e.g., culvert) that is buried in soils. To be found acceptable, the methods must each address the following issues:

- Radionuclides of interest and chosen surrogate
  - Levels and distribution of contamination
  - Internal surface condition of the piping
  - Internal residues and sediments and their radiation attenuation properties
  - Removable and fixed surface contamination
  - Instrument sensitivity and related scan and fixed minimum detectable concentrations
  - Piping geometry and presence of internally inaccessible areas/sections
  - Instrument calibration
  - Data quality objectives (DQOs)
- An industry study (Cline, J. E., "Embedded Pipe Dose Calculation Method," Electric Power Research

Institute Report No. 1000951, November, 2000) evaluated several techniques for measuring the radiological contamination on the inside of embedded pipe. Measurement techniques included pipe crawlers, gamma-ray scanners, dose rate measurements with dose-to-curie computations, scraping samples with radiochemical analyses, and smear samples with radiochemical analyses. A brief description of these methods is provided below.

The pipe crawler uses a beta sensitive detection system that is inserted into the pipe with a cable. Spacers keep the detectors at a fixed distance from the pipe wall. Measurements can be made at various points or as a continuous scan within the pipe to provide a profile of the extent and distribution of the contamination. Scaling factors based on a laboratory radiochemistry analysis of the deposited material can be applied to the measurements to provide radionuclide quantities in the pipe.

The gamma-ray scanner uses a calibrated, collimated high-purity germanium or sodium iodide spectrometer to make external measurements on the pipe. This gamma-ray scanning yields an average concentration over the length of the pipe within the field of view of the detector. The sensitivity of this method may be limited by the thickness of the piping itself and concrete between the pipe and the detector. Some radionuclide identification is possible and scaling factors can be applied as discussed above for the pipe crawler.

The dose rate measurements are also made on the external surface of the walls or floors containing the embedded pipe using a sensitive gamma detector capable of reading in the roentgen per hour range. The dose rate readings may be used directly in determining compliance with the dose criteria or used to make dose-to-curie conversions based on other measurements providing radionuclide identification.

Radionuclide identification for the contamination in the pipe may be accomplished by smear or scraping samples and radiochemical analysis. The industry report compared radionuclide ratios determined by smears and by scrapings with those found by etching the surface of the pipe. The report concluded that either of these techniques yields radionuclide mixes that are representative of the average total deposits.

Each approach is useful in specific applications, and multiple methods might be used in complex facilities like power plants. Each method also has

limitations and uncertainties that must be addressed.

Other useful information on embedded pipe characterization may be found in sources, such as the U.S. Department of Energy Innovative Technology Reports and case studies published in open literature.

Regardless of the source of the information, it is incumbent on the licensee to develop and document a comprehensive approach to embedded pipe and buried piping characterization that accounts for limitations and uncertainties, taking into account the Multi-Agency Radiation Survey and Site Investigation Manual (NUREG-1575, Rev. 1) guidance in developing the related DQOs. It should also specifically address each of the critical issues in the bulleted list above.

##### Question 9

Is the collection of additional characterization data, beyond that available from periodic radiation protection surveys, required in the license termination plan for structures, components, and soils that will be removed from the facility prior to license termination?

##### Answer to Question 9

No. In general, radiological data obtained during characterization surveys are used to determine the radiological status of the site, including facilities, buildings, surface and subsurface soils, and surface and ground water. In turn, this information is used to support the planning and design of final status surveys (FSS). In addition to providing the basis of the design of FSS, characterization surveys are used to support the following:

- Identification of remaining site dismantlement activities
- Development of new (or revisions to existing) remediation plans and procedures
- Revisions to decommissioning costs and trust fund
- Identification of environmental aspects not previously considered
- Revisions to the Environmental Report

Since the license termination process is only concerned with the status of facilities after the completion of all remediation activities, radioactivity associated with structures, components, and soils that will be removed from the facility and appropriately disposed of elsewhere, is not an issue as it cannot contribute to public dose controlled under 10 CFR 20.1402—"Radiological Criteria for Unrestricted Use." Therefore, additional characterization data need not be collected.

**Question 10**

Is characterization data required to support initial classification of Class 1 areas?

**Answer to Question 10**

Areas classified as Class 1 do not require characterization data to support that classification.

**Note:** Characterization data are needed to support decommissioning activities for all areas including:

- Determination of radionuclide distribution profiles and identification of surrogate radionuclides
- Dose modeling and development of derived concentration guideline levels
- Final status survey design and instrument selection
- Structuring the data quality objectives
- Assessment of spatial variability of radioactive contaminants on building surfaces and in surface and subsurface soils
- Assessment of whether ground water is impacted, using the results of the surface and subsurface soil characterization surveys
- Initially defining and changing the boundaries of Class 1 survey units with bordering and adjacent survey units
- Re-classification of survey units (using guidance in NUREG-1575, "Multi-Agency Radiation Survey and Site Investigation Manual," and NUREG-1727, "NMSS Decommissioning Standard Review Plan")

Dated at Rockville, MD, this 20th day of September, 2002.

For the Nuclear Regulatory Commission.

**Larry W. Camper,**

*Chief, Decommissioning Branch, Division of Waste Management, Office of Nuclear Material Safety and Safeguards.*

[FR Doc. 02-24442 Filed 9-25-02; 8:45 am]

**BILLING CODE 7590-01-P**

## **NUCLEAR REGULATORY COMMISSION**

### **Office of Nuclear Material Safety and Safeguards Consolidated Decommissioning Guidance: Decommissioning Process for Materials Licensees; Notice of Availability**

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Notice of availability.

**SUMMARY:** The Nuclear Regulatory Commission's (NRC) Office of Nuclear Material Safety and Safeguards (NMSS) is announcing the availability of a final document "Consolidated NMSS Decommissioning Guidance: Decommissioning Process for Materials Licensees" (NUREG-1757, Vol. 1). This document provides guidance for planning and implementing the termination of licenses issued through NMSS's licensing programs. The

guidance is intended for NRC staff, licensees, and the public. The guidance was developed in response to the NMSS performance goals, in the NRC's Strategic Plan, of: (1) making NRC activities and decisions more effective, efficient, and realistic; and (2) reducing unnecessary regulatory burden on stakeholders. The guidance in final NUREG-1757, Volume 1, should be used by fuel cycle, fuel storage, and materials licensees in preparing decommissioning license amendment requests, decommissioning plans, and related compliance documents. NRC staff will use the policies and procedures discussed in Volume 1 to evaluate a licensee's decommissioning actions.

**ADDRESSES:** NUREG-1757, Volume 1, is available for inspection and copying for a fee at the Commission's Public Document Room, U.S. NRC's Headquarters Building, 11555 Rockville Pike (First Floor), Rockville, Maryland. The Public Document Room is open from 7:45 a.m. to 4:15 p.m., Monday through Friday, except on Federal holidays. NUREG-1757, Volume 1, is also available electronically from the ADAMS Electronic Reading Room on the NRC Web site at: <http://www.nrc.gov/reading-rm/adams.html>, and on the NRC Web site at: <http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff>.

Copies of NUREG-1757, Volume 1, may also be purchased from one of these two sources: (1) The Superintendent of Documents, U.S. Government Printing Office, Mail Stop SSOP, Washington, DC 20402-0001; Internet: <http://bookstore.gpo.gov/>; telephone: 202-512-1800; fax: 202-512-2250; (2) The National Technical Information Service, Springfield, VA 22161-0002, Internet: [www.ntis.gov](http://www.ntis.gov); telephone 1-800-553-6847 or, locally, 703-605-6000.

**FOR FURTHER INFORMATION, CONTACT:**

Duane W. Schmidt, Mail Stop T-7F27, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001. Telephone: (301) 415-6919; Internet: [dws2@nrc.gov](mailto:dws2@nrc.gov).

**SUPPLEMENTARY INFORMATION:** As part of its redesign of the materials license program, NMSS is consolidating and updating numerous decommissioning guidance documents into a three-volume NUREG report. The three volumes are as follows: (1) Decommissioning Process for Materials Licensees; (2) Characterization, Survey, and Determination of Radiological Criteria; and (3) Financial Assurance, Recordkeeping, and Timeliness. Volume 1 of this NUREG series, entitled "Consolidated NMSS Decommissioning

Guidance: Decommissioning Process for Materials Licensees," is the first of these three volumes and is intended for use by applicants, licensees, NRC license reviewers, other NRC personnel, and Agreement State staff.

The approaches to license termination described in Volume 1 of NUREG-1757 will help to identify the information (subject matter and level of detail) needed to terminate a license by considering the wide range of radioactive materials users licensed by NRC. Volume 1 of the NUREG provides guidance for planning and implementing the termination, under the License Termination Rule (10 CFR part 20, subpart E), of licenses issued through NMSS's licensing programs. Volume 1 incorporates the risk-informed and performance-based alternatives of the rule. Volume 1 updates and builds upon the risk-informed approach in, and in part incorporates, the NMSS Decommissioning Handbook (NUREG/BR-0241, "NMSS Handbook for Decommissioning Fuel Cycle and Materials Facilities," March 1997). Volume 1 also incorporates the parts of the "NMSS Decommissioning Standard Review Plan," NUREG-1727, September 2000, that provide guidance for developing those parts of a decommissioning plan addressing general site description and current radiological conditions; decommissioning activities, management, and quality assurance; and modifications to decommissioning programs and procedures. This final Volume 1 describes and makes available to the public (1) methods acceptable to the NRC staff in implementing specific parts of the Commission's regulations; (2) techniques and criteria used by the staff in evaluating decommissioning actions; and (3) guidance to licensees responsible for decommissioning NRC-licensed sites.

The guidance in final NUREG-1757, Volume 1, should be used by fuel cycle, fuel storage, and materials licensees in preparing decommissioning license amendment requests, decommissioning plans, and related compliance documents. Other NRC licensees, e.g., uranium recovery facilities and reactors, may find this information useful, but they are not the subject of this NUREG. NRC staff will use the policies and procedures discussed in Volume 1 to evaluate a licensee's decommissioning actions. This NUREG does not substitute for regulations, and compliance with it is not required. Methods and solutions different from those in this NUREG will be acceptable, if they provide a basis for concluding that the decommissioning