

develop and submit their comments. Public comments on DG-8060 and the staff responses to the public comments are available in ADAMS under Accession No. ML22117A049.

### III. Congressional Review Act

This RG is a rule as defined in the Congressional Review Act (5 U.S.C. 801–808). However, the Office of Management and Budget has not found this RG to be a major rule as defined in the Congressional Review Act.

### IV. Backfitting, Forward Fitting, and Issue Finality

RG 8.34, Revision 1, will provide updated guidance for reactor and non-reactor applicants and licensees regarding acceptable methods for calculating radiation doses. Issuance of RG 8.34, Revision 1 would not constitute backfitting, as that term is defined in 10 CFR 50.109, “Backfitting,” and as described in NRC Management Directive (MD) 8.4, “Management of Backfitting, Forward Fitting, Issue Finality, and Information Requests”; constitute forward fitting, as that term is defined and described in MD 8.4; or affect the issue finality of any approval issued under 10 CFR part 52.

### V. Submitting Suggestions for Improvement of Regulatory Guides

A member of the public may, at any time, submit suggestions to the NRC for improvement of existing RGs or for the development of new RGs. Suggestions can be submitted on the NRC’s public website at <https://www.nrc.gov/reading-rm/doc-collections/reg-guides/contactus.html>. Suggestions will be considered in future updates and enhancements to the “Regulatory Guide” series.

Dated: August 24, 2022.

For the Nuclear Regulatory Commission.

**Meraj Rahimi,**

*Chief, Regulatory Guide and Programs Management Branch, Division of Engineering, Office of Nuclear Regulatory Research.*

[FR Doc. 2022–18525 Filed 8–26–22; 8:45 am]

**BILLING CODE 7590–01–P**

## NUCLEAR REGULATORY COMMISSION

[NRC–2022–0001]

### Sunshine Act Meetings

**TIME AND DATE:** Weeks of September 5, 12, 19, 26, October 3, 10, 2022. The schedule for Commission meetings is subject to change on short notice. The NRC Commission Meeting Schedule can be found on the internet at: <https://www.nrc.gov/public-involve/public-meetings/schedule.html>.

[www.nrc.gov/public-involve/public-meetings/schedule.html](https://www.nrc.gov/public-involve/public-meetings/schedule.html).

**PLACE:** The NRC provides reasonable accommodation to individuals with disabilities where appropriate. If you need a reasonable accommodation to participate in these public meetings or need this meeting notice or the transcript or other information from the public meetings in another format (e.g., braille, large print), please notify Anne Silk, NRC Disability Program Specialist, at 301–287–0745, by videophone at 240–428–3217, or by email at [Anne.Silk@nrc.gov](mailto:Anne.Silk@nrc.gov). Determinations on requests for reasonable accommodation will be made on a case-by-case basis.

**STATUS:** Public.

Members of the public may request to receive the information in these notices electronically. If you would like to be added to the distribution, please contact the Nuclear Regulatory Commission, Office of the Secretary, Washington, DC 20555, at 301–415–1969, or by email at [Wendy.Moore@nrc.gov](mailto:Wendy.Moore@nrc.gov) or [Tyesha.Bush@nrc.gov](mailto:Tyesha.Bush@nrc.gov).

### MATTERS TO BE CONSIDERED:

#### Week of September 5, 2022

There are no meetings scheduled for the week of September 5, 2022.

#### Week of September 12, 2022—Tentative

There are no meetings scheduled for the week of September 12, 2022.

#### Week of September 19, 2022—Tentative

*Monday, September 19, 2022*

10:00 a.m. Briefing on NRC

International Activities (Closed—Ex. 1 & 9)

#### Week of September 26, 2022—Tentative

There are no meetings scheduled for the week of September 26, 2022.

#### Week of October 3, 2022—Tentative

There are no meetings scheduled for the week of October 3, 2022.

#### Week of October 10, 2022—Tentative

*Tuesday, October 11, 2022*

10:00 a.m. NRC All Employees

Meeting (Public Meeting) (Contact: Anthony DeJesus: 301–287–9219)

*Additional Information:* The meeting will be held at the Bethesda North Marriott Hotel & Conference Center, 5701 Marinelli Road, North Bethesda, Maryland. The public is invited to attend the Commission’s meeting live by webcast at the Web address—<https://video.nrc.gov/>.

*Thursday, October 13, 2022*

9:00 a.m. Strategic Programmatic

Overview of the Operating Reactors

and New Reactors Business Lines (Public Meeting) (Contact: Jennie Rankin, 301–415–1530)

*Additional Information:* The meeting will be held in the Commissioners’ Conference Room, 11555 Rockville Pike, Rockville, Maryland. The public is invited to attend the Commission’s meeting live by webcast at the Web address—<https://video.nrc.gov/>.

### CONTACT PERSON FOR MORE INFORMATION:

For more information or to verify the status of meetings, contact Wesley Held at 301–287–3591 or via email at [Wesley.Held@nrc.gov](mailto:Wesley.Held@nrc.gov).

The NRC is holding the meetings under the authority of the Government in the Sunshine Act, 5 U.S.C. 552b.

Dated: August 24, 2022.

For the Nuclear Regulatory Commission.

**Monika G. Coflin,**

*Technical Coordinator, Office of the Secretary.*

[FR Doc. 2022–18612 Filed 8–25–22; 11:15 am]

**BILLING CODE 7590–01–P**

## NUCLEAR REGULATORY COMMISSION

[Docket No. 50–483; NRC–2022–0139]

### Union Electric Company, dba Ameren Missouri, Callaway Plant, Unit No. 1

**AGENCY:** Nuclear Regulatory Commission.

**ACTION:** Environmental assessment and finding of no significant impact; issuance.

**SUMMARY:** The U.S. Nuclear Regulatory Commission (NRC) is issuing an environmental assessment (EA) prepared under the National Environmental Policy Act of 1969 (NEPA) and NRC’s regulations. This EA summarizes the results of the NRC staff’s environmental review, which evaluates the potential environmental impacts of granting exemptions from NRC regulations and issuing an associated license amendment in response to a request from the Union Electric Company, doing business as (dba) Ameren Missouri (Ameren, the licensee) for Renewed Facility Operating License NPF–30, for the Callaway Plant, Unit No. 1 (Callaway). Specifically, the licensee is seeking a license amendment and regulatory exemptions that would, if granted, allow the licensee to use both a deterministic and risk-informed approach to address safety issues discussed in Generic Safety Issue (GSI)–191, “Assessment of Debris Accumulation in PWR [Pressurized Water Reactor] Sump Pump Performance” and to close Generic

Letter (GL) 2004–02, “Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at Pressurized-Water Reactors.” The NRC staff is issuing a final EA and finding of no significant impact (FONSI) associated with the proposed exemptions.

**DATES:** The EA and FONSI referenced in this document is available on August 29, 2022.

**ADDRESSES:** Please refer to Docket ID NRC–2022–0139 when contacting the NRC about the availability of information regarding this document. You may obtain publicly available information related to this document using any of the following methods:

- **Federal Rulemaking Website:** Go to <https://www.regulations.gov> and search for Docket ID NRC–2022–0139. Address questions about Docket IDs in [Regulations.gov](https://www.regulations.gov) to Stacy Schumann; telephone: 301–415–0624; email: [Stacy.Schumann@nrc.gov](mailto:Stacy.Schumann@nrc.gov). For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION**

**CONTACT** section of this document.

- **NRC’s Agencywide Documents Access and Management System (ADAMS):** You may obtain publicly available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/adams.html>. To begin the search, select “Begin Web-based ADAMS Search.” For problems with ADAMS, please contact the NRC’s Public Document Room (PDR) reference staff at 1–800–397–4209, 301–415–4737, or by email to [PDR.Resource@nrc.gov](mailto:PDR.Resource@nrc.gov). For the convenience of the reader, instructions about obtaining materials referenced in this document are provided in the “Availability of Documents” section.

- **NRC’s PDR:** You may examine and purchase copies of public documents, by appointment, at the NRC’s PDR, Room P1 B35, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852. To make an appointment to visit the PDR, please send an email to [PDR.Resource@nrc.gov](mailto:PDR.Resource@nrc.gov) or call 1–800–397–4209 or 301–415–4737, between 8:00 a.m. and 4:00 p.m. Eastern Time (ET), Monday through Friday, except Federal holidays.

**FOR FURTHER INFORMATION CONTACT:** Mahesh Chawla, Office of Nuclear Reactor Regulation, U.S. Nuclear Regulatory Commission, Washington, DC 20555–0001; telephone: 301–415–8371, email: [Mahesh.Chawla@nrc.gov](mailto:Mahesh.Chawla@nrc.gov).

#### **SUPPLEMENTARY INFORMATION:**

##### **I. Introduction**

The NRC is considering a request to grant certain regulatory exemptions and

issue a license amendment for Renewed Facility Operating License NPF–30, issued to Ameren, for Callaway, located in Callaway County, Missouri. The regulatory exemptions and associated license amendment, if granted, would allow Ameren to incorporate the use of a risk-informed approach to address safety issues discussed in GSI–191 and respond to GL 2004–02. Pursuant to Section 51.21 of title 10 of the *Code of Federal Regulations* (10 CFR), “Criteria for and identification of licensing and regulatory actions requiring environmental assessments,” the NRC has prepared an EA summarizing the findings of the NEPA review of the proposed action. The NRC concluded that the proposed action will have no significant environmental impact. In accordance with 10 CFR 51.31(a), the NRC has determined not to prepare an environmental impact statement for the proposed licensing actions and is issuing a FONSI.

The NRC established GSI–191 to determine whether the transport and accumulation of debris from a loss-of-coolant accident (LOCA) in the PWR containment structure would impede the operation of the emergency core cooling system (ECCS) or containment spray system (CSS). A LOCA within the containment structure is assumed to be caused by a break in the primary coolant loop piping. Water discharged from the pipe break and debris would collect on the containment structure floor and within the containment emergency sump. During this type of accident, the ECCS and CSS would initially draw cooling water from the refueling water storage tank. However, realigning the ECCS pumps to the containment emergency sump would provide long-term cooling of the reactor core. Therefore, successful long-term cooling depends on the ability of the containment emergency sump to provide adequate flow to the residual heat removal (RHR) recirculation pumps for extended periods of time.

One of the concerns addressed by the implementation of GSI–191 is that debris material, such as insulation installed on piping and components, within the containment structure, could be dislodged by a jet of high-pressure water and steam during the LOCA. Water, along with debris, would accumulate at the bottom of the containment structure and flow towards the emergency sump pumps. Insulation and other fibrous debris material could block the emergency sump screens and suction strainers, which in turn could prevent the ability of the containment emergency sump to provide adequate water flow to the RHR pumps (for more

information, see NUREG–0897, “Containment Emergency Sump Performance: Technical Finding Related to Unresolved Safety Issue A–43,” Revision 1).

The NRC issued GL 2004–02 to address this safety concern by requesting PWR licensees, pursuant to 10 CFR 50.54(f), to use an NRC-approved methodology to perform a “mechanistic evaluation of the potential for the adverse effects of post-accident debris blockage and operation with debris-laden fluids to impede or prevent the recirculation functions of the ECCS and CSS following all postulated accidents for which the recirculation of these systems is required” and submit this information to the NRC for evaluation.

In 2012, the NRC staff developed options for resolution of GSI–191, which are discussed in SECY–12–0093, “Closure Options for Generic Safety Issue 191, Assessment of Debris Accumulation on Pressurized-Water Reactor Sump Performance,” dated July 9, 2012. The licensee has proposed to use both a deterministic method, with plant-specific testing, and a risk-informed approach to demonstrate compliance with 10 CFR 50.46, “Acceptance criteria for emergency core colling systems for light-water nuclear power reactors,” and 10 CFR part 50, appendix A, General Design Criteria (GDC) 35, “Emergency core cooling,” GDC 38, “Containment heat removal,” and GDC 41, “Containment atmosphere cleanup,” and to resolve GSI–191 for Callaway. Because, historically, the NRC staff has not allowed licensees to use a risk-informed approach to show compliance with the requirements of 10 CFR 50.46, the licensee requested exemptions from 10 CFR 50.46(a)(1) and GDC 35, 38, and 41, as well as an amendment to the associated technical specifications to allow the use of a risk-informed approach to resolve GSI–191. If approved, the proposed action would not authorize any modifications within the containment structure, physical changes to the ECCS, or other modifications to the plant. Rather, the proposed action would only allow the use of an alternate methodology to show compliance with the regulations that require the ECCS and CSS function during certain LOCA events.

##### **II. Environmental Assessment**

###### *Description of the Proposed Action*

The proposed action as requested by the licensee is to grant certain regulatory exemptions and amend Facility Operating License NPF–30. The regulatory exemptions would allow

Ameren to change the licensing basis LOCA analysis identified in the updated final safety analysis report to use a risk-informed approach to address safety issues discussed in GSI-191 and to close GL 2004 02. If approved, no physical modifications to the nuclear plant or changes to reactor operations involving the ECCS would be required. The proposed action is in response to the licensee's application dated March 31, 2021, as supplemented by letters dated May 27, 2021; July 22, 2021; August 23, 2021; October 7, 2021; January 27, 2022; March 8, 2022; and May 26, 2022.

#### *Need for the Proposed Action*

The proposed action is needed because, as the holder of Renewed Facility Operating License No. NPF-30, Ameren is expected to address the safety issues discussed in GSI-191 and to close GL 2004-02 for Callaway. Consistent with SECY-12-0093, the licensee chose an approach, which requires, in part, that Ameren request that the NRC amend the renewed facility operating license and grant certain regulatory exemptions for Callaway.

#### *Environmental Impacts of the Proposed Action*

Callaway is located on an approximately 7,354-acre (2,976 hectare) site in Callaway County, Missouri, approximately 10 miles (16 kilometers) southeast of Fulton, Missouri, and 80 miles (129 kilometers) west of the St. Louis metropolitan area.

Callaway consists of a single four-loop Westinghouse PWR unit. The reactor core of the unit heats water, which is pumped to four steam generators, where the heated water is converted to steam. The steam is then used to turn turbines, which are connected to electrical generators that produce electricity. A simplified drawing of a PWR can be viewed at <https://www.nrc.gov/reactors/pwrs.html>.

The reactor, steam generators, and other components are housed in a concrete and steel containment structure (building). The containment structure is a reinforced concrete cylinder with a concrete slab base and hemispherical dome. A welded steel liner is attached to the inside face of the concrete shell to ensure a high degree of leak tightness. In addition, the 4-foot (1.2-meter)-thick concrete walls of the containment structure serve as a radiation shield. Additional information on the plant structures and systems, as well as the environmental impact statement for license renewal, can be found in NUREG-1437, Supplement 51,

“Generic Environmental Impact Statement for License Renewal of Nuclear Plants: Supplement 51 Regarding Callaway Plant, Unit 1: Final Report.”

Radiological and non-radiological impacts on the environment that may result from granting the regulatory exemptions and issuing the license amendment are summarized in the following sections.

#### *Non-Radiological Impacts*

No changes would be made to structures or land use within the Callaway site as a result of the proposed action, and non-radiological liquid effluents or gaseous emissions would not change. In addition, the license amendment and regulatory exemptions would not result in any changes to the use of resources or create any new environmental impacts. Therefore, there would be no non-radiological impacts to environmental resources or any irreversible and irretrievable commitments.

Since granting the regulatory exemptions and issuing the license amendment would not result in environmental effects, there would be no non-radiological cumulative impact.

#### *Radiological Impacts*

##### *Radioactive Gaseous and Liquid Effluents and Solid Waste*

Callaway uses waste treatment systems to collect, process, recycle, and dispose of gaseous, liquid, and solid wastes that contain radioactive material in a safe and controlled manner within NRC and Environmental Protection Agency (EPA) radiation safety standards.

The license amendment and regulatory exemptions, if granted, would not require any physical change to the nuclear plant or reactor operations; therefore, there would be no changes to the plant radioactive waste treatment systems. A detailed description of the Callaway radioactive waste handling and disposal activities is presented in chapter 2.1.2 of Supplement 51 to NUREG-1437.

##### *Radioactive Gaseous Effluents*

The objectives of the Callaway gaseous waste management system (GWMS) are to process and control the release of radioactive gaseous effluents into the environment to be within the requirements of 10 CFR 20.1301, “Dose limits for individual members of the public,” and to be consistent with the as low as reasonably achievable (ALARA) dose objectives set forth in Appendix I to 10 CFR part 50. The GWMS is designed so that radiation exposure to

plant workers is within the dose limits in 10 CFR 20.1201, “Occupational dose limits for adults.”

Granting the regulatory exemptions and issuing the license amendment would not require any physical changes to the nuclear plant or reactor operations that would affect the release of radioactive gaseous effluents into the environment; therefore, there would be no changes to the GWMS. The existing equipment and plant procedures that control radioactive releases to the environment would continue to be used to maintain radioactive gaseous releases within the dose limits in 10 CFR 20.1301 and the ALARA dose objectives in Appendix I to 10 CFR part 50.

##### *Radioactive Liquid Effluents*

The function of the Callaway liquid waste processing system (LWPS) is to collect and process radioactive liquid wastes to reduce radioactivity and chemical concentrations to levels acceptable for discharge to the environment or to recycle the liquids for use in plant systems. The principal objectives of the LWPS are to collect liquid effluents (wastes) that may contain radioactive material and to maintain sufficient processing capability so that liquid waste may be discharged to the environment below the regulatory limits in 10 CFR 20.1301 and consistent with the ALARA dose objectives in Appendix I to 10 CFR part 50. The liquid effluent is routed through a monitor that measures the radioactivity and can automatically terminate the release in the event radioactivity exceeds predetermined levels. The liquid effluent is discharged from the plant into the Missouri River via a pipeline.

Granting the regulatory exemptions and issuing the license amendment would not require any physical change to the nuclear plant or reactor operations; therefore, there would be no changes to the LWPS. The existing equipment and plant procedures that control radioactive releases to the environment will continue to be used to maintain radioactive liquid releases within the dose limits in 10 CFR 20.1301 and the ALARA dose objectives in Appendix I to 10 CFR part 50.

##### *Radioactive Solid Wastes*

The function of the Callaway solid waste processing system (SWPS) is to process, package, and store the solid radioactive wastes generated by nuclear plant operations until they are shipped off site to a vendor for further processing or for permanent disposal at a licensed burial facility, or both. The storage areas have restricted access and

shielding to reduce radiation rates to plant workers. The principal objectives of the SWPS are to package and transport the waste in compliance with NRC regulations in 10 CFR part 61, "Licensing Requirements for Land Disposal of Radioactive Waste," and 10 CFR part 71, "Packaging and Transportation of Radioactive Material," and the U.S. Department of Transportation regulations in 49 CFR parts 170 through 179; and to maintain the dose limits in 10 CFR 20.1201, 10 CFR 20.1301, and Appendix I to 10 CFR part 50.

The existing equipment and plant procedures that control radioactive solid waste handling would continue to be used to maintain exposures within the dose limits in 10 CFR 20.1201, 10 CFR 20.1301, and 10 CFR part 50 appendix I. Therefore, there will be no changes to the SWPS and issuing the license amendment and granting the regulatory exemptions will not result in any physical changes to the nuclear plant or reactor operations that would affect the release of radioactive solid wastes into the environment.

#### Occupational Radiation Doses

The license amendment and regulatory exemptions, if granted, would not require any physical change to the nuclear plant or changes to reactor operations; therefore, there would be no change to any in-plant radiation sources. In addition, no new operator actions would be implemented that could affect occupational radiation exposure. The licensee's radiation protection program monitors radiation levels throughout the nuclear plant to establish appropriate work controls, training, temporary shielding, and protective equipment requirements so that worker doses remain within the dose limits in 10 CFR part 20, "Standards for Protection Against Radiation," subpart C, "Occupational Dose Limits." The license amendment and regulatory exemptions would not change radiation levels within the nuclear plant and, therefore, there would be no increased radiological impact to the workers.

#### Offsite Radiation Dose

The primary sources of offsite dose to members of the public from Callaway are radioactive gaseous and liquid effluents. As discussed previously, there would be no change to the operation of Callaway radioactive GWMS and LWPS or their ability to perform their intended functions. Also, there would be no change to the Callaway radiation monitoring system and procedures used to control the release of radioactive

effluents in accordance with radiation protection standards in 10 CFR 20.1301, 40 CFR part 190, "Environmental Radiation Protection Standards for Nuclear Power Operations," and the ALARA dose objectives in appendix I to 10 CFR part 50.

Based on this information, the offsite radiation dose to members of the public would not change and would continue to be within regulatory limits. Therefore, the license amendment and regulatory exemptions would not change offsite dose levels and, consequently, there would be no significant health effects from the proposed action.

#### Design-Basis Accidents

Design-basis accidents at Callaway, are evaluated by both the licensee and the NRC to ensure that the unit would continue to withstand the spectrum of postulated accidents without undue hazard to the public health and safety and to ensure the protection of the environment.

Separate from its environmental review, the NRC is evaluating the licensee's technical and safety analyses provided in support of the proposed action. The results of the NRC staff's safety review and conclusion will be documented in a publicly available safety evaluation. The NRC staff must conclude in its safety evaluation that taking the proposed action will (1) provide reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) provide reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) not be inimical to the common defense and security or to the health and safety of the public. The NRC will not take the proposed action absent such a safety conclusion.

#### Radiological Cumulative Impacts

The radiological dose limits for protection of the public and plant workers have been developed by the NRC and the EPA to address the cumulative impact of acute and long-term exposure to radiation and radioactive material. These dose limits are codified in 10 CFR part 20 and 40 CFR part 190.

Cumulative radiation doses are required to be within the limits set forth in the regulations cited in the previous paragraph. The license amendment and exemptions would not require physical changes to the plant or changes to plant activities; in-plant radiation sources would not change and offsite radiation dose to members of the public would

not change. Therefore, the NRC staff concludes that there would be no significant cumulative radiological impact from the proposed action.

#### Radiological Impacts Summary

Based on these evaluations, the license amendment and exemptions would not result in any significant radiological impacts. Therefore, the safety evaluation must conclude that the proposed action will (1) provide reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) provide reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) not be inimical to the common defense and security or to the health and safety of the public. The NRC would not take the proposed action absent such a safety conclusion.

#### *Environmental Impacts of the Alternatives to the Proposed Action*

As discussed earlier, licensees have options for responding to GL 2004-02 and for demonstrating compliance with 10 CFR 50.46. Consistent with these options and as an alternative to the proposed action, the licensee could choose to remove and replace insulation within the reactor containment building. This alternative would require the physical removal and disposal of significant amounts of insulation from a radiation area within the reactor containment building, and the installation of new insulation less likely to impact sump performance.

The removal of the existing insulation from the containment building would generate radiologically contaminated waste. Ameren estimated that approximately 5,500 cubic feet (6.6 tons) of fiberglass insulation would have to be removed from the Callaway containment. The removed insulation would require special handling and packaging so that it could be safely transported from the site. The licensee would likely use existing facilities to process and store this material until it could be transported to a low-level radioactive or hazardous waste disposal site. Energy (fuel) would be expended to transport the insulation and land would be expended at the disposal site.

The removal of the old insulation and installation of new insulation would expose workers to radiation. Based on planning documents prepared in 2010, Ameren estimated that the expected total dose for replacing insulation in Callaway, would be between 350 and 400 person-rem. This estimate was considered in line with estimates from

other utilities impacted by this same issue. Ameren also indicated that this initial estimate would now likely be higher due to the intervening 12 years of continuous plant operation. The NRC reviewed NUREG-0713, Volume 41, "Occupational Radiation Exposure at Commercial Nuclear Power Reactors and Other Facilities: Fifty-Second Annual Report," and determined that Ameren's average baseline collective radiation exposure is approximately 22 person-rem. The additional 350 to 400-plus person-rem collective exposure would be shared across the entire work force involved with removing and reinstalling insulation. In SECY-12-0093, the NRC staff attempted to develop a total occupational dose estimate for the work involved in insulation removal and replacement associated with GSI-191. Due to uncertainties in the scope of work required to remove and replace insulation at a specific nuclear plant and other site-specific factors such as source term and hazardous materials, the NRC staff was unable to estimate the total occupational dose associated with this work. However, dose estimates were provided by the Nuclear Energy Institute (NEI) in a letter to the NRC dated March 30, 2012, based on information collected on occupational radiation exposures that have been, or could be, incurred during insulation removal and replacement. In the letter, NEI noted similar difficulties in estimating the potential amount of radiation exposure, but provided a "per unit" estimate of between 80 and 525 person-rem. Given uncertainties in the scope of work and other nuclear plant-specific factors such as source term and hazardous materials, the NRC staff found no basis to conclude that the NEI estimates were unreasonable.

Accordingly, because Ameren's estimate of potential additional radiation exposure resulting from the alternative approach of removing and replacing insulation is consistent with the NEI estimated range, the NRC staff considers Ameren's estimate to be reasonable.

As stated in the "Occupational Radiation Doses" section of this document, Ameren's radiation protection program monitors radiation

levels throughout the nuclear plant to establish appropriate work controls, training, temporary shielding, and protective equipment requirements so that worker doses are expected to remain within the dose limits in 10 CFR 20.1201.

In addition, as stated in the "Offsite Radiation Dose" section of this document, Ameren also has a radiation monitoring system and procedures in place to control the release of radioactive effluents in accordance with radiation protection standards in 10 CFR 20.1301, 40 CFR part 190, and the ALARA dose objectives in appendix I to 10 CFR part 50. Therefore, radiation exposure to members of the public would be maintained within the NRC dose criteria in 10 CFR 20.1301, 40 CFR part 190, and the ALARA dose objectives of appendix I to 10 CFR part 50.

Based on this information, impacts to members of the public from removing and replacing insulation within the reactor containment building would not be significant. However, impacts to plant workers and the environment from implementing this alternative would be greater than implementing the proposed action.

#### *Alternative Use of Resources*

The proposed action would not involve the use of any different resources (e.g., water, air, land, nuclear fuel) not previously considered in NUREG-1437, Supplement 51.

#### *Agencies and Persons Consulted*

In accordance with its stated policy, on June 27, 2022, the NRC staff consulted with the State of Missouri official, Mr. Aaron Schmidt, regarding the environmental impact of the proposed action. The State of Missouri official has not provided any comments on the EA and FONSI.

### **III. Finding of No Significant Impact**

The licensee requested to amend Renewed Facility Operating License No. NPF-30 to grant exemptions for Callaway, from certain requirements of 10 CFR 50.46(a)(1) and 10 CFR part 50, appendix A, GDC 35, 38, and 41. This proposed action would not significantly affect plant safety, would not have a

significant adverse effect on the probability of an accident occurring, and would not have any significant radiological or non-radiological impacts. It would also not result in any changes to radioactive effluents or emissions, exposures to nuclear plant workers and members of the public, or any changes to radiological and non-radiological impacts to the environment.

Consistent with 10 CFR 51.21, the NRC conducted an environmental review of the proposed action. Based on the EA included in Section II of this notice and incorporated by reference in this FONSI, the NRC staff finds that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the NRC has determined there is no need to prepare an environmental impact statement for the proposed action.

The NRC staff's evaluation considered the information provided in the licensee's application as supplemented, and the NRC staff's review of related environmental documents. Section IV of this notice lists documents related to the proposed action and includes information on the availability of the documents, including the related environmental document NUREG-1437, Supplement 51, which provides the latest environmental review of current operations and description of environmental conditions at Callaway.

This FONSI and other related environmental documents may be examined, and/or copied for a fee, at the NRC's PDR, located at One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852. Publicly available records are also accessible online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/adams.html>. Persons who do not have access to ADAMS or who encounter problems in accessing the documents located in ADAMS should contact the NRC's PDR reference staff by telephone at 1-800-397-4209 or 301-415-4737, or by email to [PDR.Resource@nrc.gov](mailto:PDR.Resource@nrc.gov).

### **IV. Availability of Documents.**

The documents identified in the following table are available to interested persons through one or more of the following methods, as indicated.

Document	ADAMS accession No.
U.S. Nuclear Regulatory Commission. Generic Letter 2004-02: "Potential Impact of Debris Blockage on Emergency Recirculation During Design Basis Accidents at Pressurized-Water Reactors," dated September 13, 2004.	ML042360586
U.S. Nuclear Regulatory Commission. NUREG-0897, "Containment Emergency Sump Performance: Technical Findings Related to Unresolved Safety Issue A-43, Revision 1, October 1985.	ML112440046
Ameren Missouri. Letter ULNRC-06526, "Request for License Amendment and Regulatory Exemptions for a Risk-Informed Approach to Address GSI-191 and Respond to GL 2004-02," dated March 31, 2021.	ML21090A184 (package).

Document	ADAMS accession No.
Ameren Missouri. Letter ULNRC-06664, "Supplement to Request for License Amendment and Regulatory Exemptions for a Risk-Informed Approach to Address GSI-191 and Respond to GL 2004-02 (LDCN 19-0014)," dated May 27, 2021.	ML21147A222
Ameren Missouri. Letter ULNRC-06651, "Supplement to Request for License Amendment and Regulatory Exemptions for a Risk-Informed Approach to Address GSI-191 and Respond to GL 2004-02 (LDCN 19-0014)," dated July 22, 2021.	ML21203A192 (package).
Ameren Missouri. Letter ULNRC-06683, "Transmittal of Documents Identified from NRC Audit of License Amendment Request Regarding Risk-Informed Approach to Closure of Generic Safety Issue 191 (EPID L-2021-LLA-0059)," dated August 23, 2021.	ML21237A135 (package).
Ameren Missouri. Letter ULNRC-06692, "Third Supplement to Request for License Amendment and Regulatory Exemptions for a Risk-Informed Approach to Address GSI-191 and Respond to GL 2004-02 (LDCN 19-0014)," dated October 7, 2021.	ML21280A378 (package).
Ameren Missouri. Letter ULNRC-06690, "Fourth (Post-Audit) Supplement to Request for License Amendment and Regulatory Exemptions for a Risk-Informed Approach to Address GSI-191 and Respond to GL 2004-02 (LDCN 19-0014)," dated January 27, 2022.	ML22027A804 (package).
Ameren Missouri. Letter ULNRC-06721, "Fifth (Post-Audit) Supplement to Request for License Amendment and Regulatory Exemptions for a Risk-Informed Approach to Address GSI-191 and Respond to GL 2004-02 (LDCN 19-0014)," dated March 8, 2022.	ML22068A027 (package).
Ameren Missouri. Letter ULNRC-06735, "Response to Request for Additional Information Regarding Request for License Amendment and Regulatory Exemptions for Risk-Informed Approach to Address GSI-191 and Respond to Generic Letter 2004-02," dated May 26, 2022.	ML22146A337 (package).
Nuclear Energy Institute. GSI-191 Dose Estimates, dated March 30, 2012 .....	ML12095A319
SECY-12-0093, "Closure Options for Generic Safety Issue—191, Assessment of Debris Accumulation on Pressurized-Water Reactor Sump Performance," dated July 9, 2012.	ML121320270 (package).
SRM-SECY-12-0093, "Staff Requirements—SECY-12-0093—Closure Options for Generic Safety Issue—191, Assessment of Debris Accumulation on Pressurized-Water Reactor Sump Performance," dated December 14, 2012.	ML12349A378
U.S. Nuclear Regulatory Commission. NUREG-1437, Supplement 51, "Generic Environmental Impact Statement for License Renewal of Nuclear Plants: Supplement 51 Regarding Callaway Plant, Unit 1: Final Report," October 2014.	ML14289A140
U.S. Nuclear Regulatory Commission. NUREG-0713, Volume 41, "Occupational Radiation Exposure at Commercial Nuclear Power Reactors and Other Facilities 2019: Fifty-Second Annual Report," April 2022.	ML22111A013

Dated: August 23, 2022.

For the Nuclear Regulatory Commission.

**Siva P. Lingam,**

*Project Manager, Plant Licensing Branch IV,  
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of Nuclear Reactor Regulation.*

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## **PENSION BENEFIT GUARANTY CORPORATION**

### **Proposed Submission of Information Collection for OMB Review; Comment Request; Annual Reporting (Form 5500 Series)**

**AGENCY:** Pension Benefit Guaranty Corporation.

**ACTION:** Notice of intent to request extension of OMB approval of information collection.

**SUMMARY:** The Pension Benefit Guaranty Corporation (PBGC) intends to request that the Office of Management and Budget (OMB) extend approval, with modifications, under the Paperwork Reduction Act, of a collection of information for Annual Reporting under OMB control number 1212-0057, which expires on June 30, 2025. This notice informs the public of PBGC's intent and solicits public comment on the collection of information.

**DATES:** Comments must be submitted on or before October 28, 2022.

**ADDRESSES:** Comments may be submitted by any of the following methods:

- *Federal eRulemaking Portal:* <http://www.regulations.gov>. Follow the online instructions for submitting comments.
- *Email:* [paperwork.comments@pbgc.gov](mailto:paperwork.comments@pbgc.gov). Refer to OMB control number 1212-0057 in the subject line.
- *Mail or Hand Delivery:* Regulatory Affairs Division, Office of the General Counsel, Pension Benefit Guaranty Corporation, 445 12th Street SW, Washington, DC 20024-2101.

Commenters are strongly encouraged to submit public comments electronically. PBGC expects to have limited personnel available to process public comments that are submitted on paper through mail. Until further notice, any comments submitted on paper will be considered to the extent practicable.

All submissions must include the agency's name (Pension Benefit Guaranty Corporation, or PBGC) and refer to OMB control number 1212-0057. Comments received will be posted without change to PBGC's website, [www.pbgc.gov](http://www.pbgc.gov), including any personal information provided. Do not submit comments that include any personally identifiable information or confidential business information.

Copies of the collection of information may be obtained by writing to Disclosure Division, Office of the General Counsel, Pension Benefit Guaranty Corporation, 445 12th Street SW, Washington, DC 20024-2101, or calling 202-229-4040 during normal business hours. If you are deaf or hard of hearing, or have a speech disability, please dial 7-1-1 to access telecommunications relay services.

**FOR FURTHER INFORMATION CONTACT:** Karen Levin ([levin.karen@pbgc.gov](mailto:levin.karen@pbgc.gov)), Attorney, Regulatory Affairs Division, Office of the General Counsel, Pension Benefit Guaranty Corporation, 445 12th Street SW, Washington, DC 20024-2101, 202-229-3559. If you are deaf or hard of hearing, or have a speech disability, please dial 7-1-1 to access telecommunications relay services.

**SUPPLEMENTARY INFORMATION:** Annual reporting to the Internal Revenue Service (IRS), the Employee Benefits Security Administration (EBSA), and the Pension Benefit Guaranty Corporation (PBGC) is required by law for most employee benefit plans. For example, section 4065 of the Employee Retirement Income Security Act of 1974 (ERISA) requires annual reporting to PBGC for pension plans covered by title IV of ERISA. To accommodate these filing requirements, IRS, EBSA, and PBGC have jointly promulgated the Form 5500 Series, which includes the