

holders) will need to show a valid, officially-issued picture identification at the gate to enter the NASA Research Park. For questions, please call Ms Irma Rodriguez at (202) 358-0984. It is imperative that these meetings be held on this date to accommodate the scheduling priorities of the key participants.

**Patricia D. Rausch,**

*Advisory Committee Management Officer,  
National Aeronautics and Space  
Administration.*

[FR Doc. 2016-26145 Filed 10-28-16; 8:45 am]

**BILLING CODE 7510-13-P**

## NATIONAL CREDIT UNION ADMINISTRATION

### Submission for OMB Review; Comment Request

**AGENCY:** National Credit Union  
Administration (NCUA).

**ACTION:** Notice.

**SUMMARY:** The National Credit Union Administration (NCUA) will be submitting the following information collection requests to the Office of Management and Budget (OMB) for review and clearance in accordance with the Paperwork Reduction Act of 1995, Public Law 104-13, on or after the date of publication of this notice.

**DATES:** Comments should be received on or before November 30, 2016 to be assured of consideration.

**ADDRESSES:** Send comments regarding the burden estimate, or any other aspect of the information collection, including suggestions for reducing the burden, to (1) Office of Information and Regulatory Affairs, Office of Management and Budget, Attention: Desk Officer for NCUA, New Executive Office Building, Room 10235, Washington, DC 20503, or email at [OIRA\\_Submission@OMB.EOP.gov](mailto:OIRA_Submission@OMB.EOP.gov) and (2) NCUA PRA Clearance Officer, 1775 Duke Street, Alexandria, VA 22314, Suite 5067, or email at [PRAComments@ncua.gov](mailto:PRAComments@ncua.gov).

**FOR FURTHER INFORMATION CONTACT:** Copies of the submission may be obtained by emailing [PRAComments@ncua.gov](mailto:PRAComments@ncua.gov) or viewing the entire information collection request at [www.reginfo.gov](http://www.reginfo.gov).

### SUPPLEMENTARY INFORMATION:

*OMB Number:* 3133-0033.

*Title:* Security Program, 12 CFR 748.

*Abstract:* In accordance with Title V of the Gramm-Leach-Bliley Act (15 U.S.C. 6801 *et seq.*), as implemented by 12 CFR part 748, federally-insured credit unions (FICU) are required to develop and implement a written

security program to safeguard sensitive member information. This information collection requires that such programs be designed to respond to incidents of unauthorized access or use, in order to prevent substantial harm or serious inconvenience to members.

*Type of Review:* Extension of a previously approved collection.

*Affected Public:* Private Sector: Not-for-profit institutions.

*Estimated Total Annual Burden Hours:* 15,982.

*OMB Number:* 3133-0168.

*Title:* Maximum Borrowing Authority, 12 CFR 741.2.

*Abstract:* Section 741.2 of the NCUA Rules and Regulations (12 CFR 741.2) places a maximum borrowing limitation on federally insured credit unions of 50 percent of paid-in and unimpaired capital and surplus. This limitation is statutory for federal credit unions. The collection of information requirement is for federally insured state-chartered credit unions seeking a waiver from the borrowing limit. These credit unions must submit a detailed safety and soundness analysis, a proposed aggregate amount, a letter from the state regulator approving the request and an explanation of the need for the waiver to the NCUA Regional Director. This collection of information is necessary to protect the National Credit Union Share Insurance Fund ("Fund"). The NCUA must be made aware of and be able to monitor those credit unions seeking a waiver from the maximum borrowing limitation.

*Type of Review:* Extension without change of a previously approved collection.

*Affected Public:* Private Sector: Not-for-profit institutions.

*Estimated Total Annual Burden Hours:* 16.

By Gerard Poliquin, Secretary of the Board, the National Credit Union Administration, on October 19, 2016.

Dated: October 26, 2016.

**Dawn D. Wolfgang,**

*NCUA PRA Clearance Officer.*

[FR Doc. 2016-26169 Filed 10-28-16; 8:45 am]

**BILLING CODE 7535-01-P**

## NUCLEAR REGULATORY COMMISSION

[NRC-2015-0160]

### NuScale Power, LLC, Design-Specific Review Standard and Scope and Safety Review Matrix

**AGENCY:** Nuclear Regulatory  
Commission.

**ACTION:** NuScale design-specific review standard; issuance.

**SUMMARY:** The U.S. Nuclear Regulatory Commission (NRC or Commission) has issued the NuScale Power, LLC, (NuScale), Design-Specific Review Standard (DSRS) Sections, and is issuing the final NuScale DSRS Scope and Safety Review Matrix, for NuScale Design Certification (DC), Combined License (COL), and Early Site Permit (ESP) reviews. The NRC staff is also issuing the DSRS public comment resolution matrices, which address the comments received on the draft DSRS. The NuScale DSRS provides guidance to the NRC staff for performing safety reviews for those specific areas where existing NUREG-0800, "Standard Review Plan [SRP] for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR Edition," sections do not address the unique features of the NuScale design.

**DATES:** The DSRS sections were effective upon issuance between June 24 and August 4, 2016.

**ADDRESSES:** Please refer to Docket ID NRC-2015-0160 when contacting the NRC about the availability of information regarding this document. You may obtain publically-available information related to this document using any of the following methods:

- *Federal Rulemaking Web site:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2015-0160. Address questions about NRC dockets to Carol Gallagher; telephone: 301-415-3463; email: [Carol.Gallagher@nrc.gov](mailto:Carol.Gallagher@nrc.gov). For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly-available documents online in the ADAMS Public Documents collection at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov). The ADAMS accession number for each document referenced (if it is available in ADAMS) is provided the first time that it is mentioned in this document. The DSRS is available in ADAMS Package Accession No. ML15355A295 and the final NuScale DSRS Scope and Safety Review Matrix is also available in ADAMS under Accession No. ML16263A000. The resolution of

comments on the draft DSRS is documented in the DSRS Public Comment Resolution Matrices (ADAMS Package Accession No. ML16083A615). In addition, for the convenience of the reader, the ADAMS accession numbers are provided in a table in the "Availability of Documents" section of this document.

- *NRC's PDR*: You may examine and purchase copies of public documents at the NRC's PDR, Room O1-F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

**FOR FURTHER INFORMATION CONTACT:**

Rajender Auluck, telephone: 301-415-1025; email: [Rajender.Auluck@nrc.gov](mailto:Rajender.Auluck@nrc.gov) or Gregory Cranston, telephone: 301-415-0546; email: [Gregory.Cranston@nrc.gov](mailto:Gregory.Cranston@nrc.gov); both are staff members of the Office of New Reactors, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

**SUPPLEMENTARY INFORMATION:**

**I. Background**

In the Staff Requirements Memorandum (SRM) COMGBJ-10-0004/COMGEA-10-0001, "Use of Risk Insights to Enhance Safety Focus of Small Modular Reactor Reviews," dated August 31, 2010 (ADAMS Accession No. ML102510405), the Commission provided direction to the NRC staff on the preparation for, and review of, small modular reactor (SMR) applications, with a near-term focus on integral pressurized-water reactor designs. The Commission directed the NRC staff to more fully integrate the use of risk insights into pre-application activities and the review of applications and, consistent with regulatory requirements and Commission policy statements, to align the review focus and resources to risk-significant structures, systems, and components and other aspects of the design that contribute most to safety in order to enhance the effectiveness and efficiency of the review process. The Commission directed the NRC staff to develop a design-specific, risk-informed review plan for each SMR design to address pre-application and application review activities. An important part of this review plan is the DSRS. The DSRS for the NuScale design is the result of the implementation of the Commission's direction.

**II. DSRS for the NuScale Design**

The NuScale DSRS (available in ADAMS Package Accession No. ML15355A295) reflects current NRC staff safety review methods and practices which integrate risk insights and, where appropriate, lessons learned from the NRC's reviews of DC and COL

applications completed since the last revision of the NUREG-0800, SRP Introduction, Part 2, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: Light-Water Small Modular Reactor Edition," January 2014 (ADAMS Accession No. ML13207A315). The NuScale DSRS Scope and Safety Review Matrix provides a complete list of SRP sections and identifies which SRP sections will be used for DC, COL, or ESP reviews concerning the NuScale design; which SRP sections are not applicable to the NuScale design; which SRP sections needed modification and were reissued as DSRS sections; and which new DSRS sections were added to address a unique design consideration in the NuScale design. The final NuScale DSRS Scope and Safety Review Matrix is available in ADAMS under Accession No. ML16263A000.

The NRC staff developed the content of the NuScale DSRS as an alternative method for evaluating a NuScale-specific application and has determined that the application may address the DSRS in lieu of addressing the SRP, with specified exceptions. These exceptions include particular review areas in which the DSRS directs reviewers to consult the SRP and others in which the SRP is used for the review as identified in the final NuScale DSRS Scope and Safety Review Matrix. If NuScale chooses to address the DSRS, the application should identify and describe all differences between the design features (DC and COL applications only), analytical techniques, and procedural measures proposed in an application and the guidance of the applicable DSRS section (or SRP section, as specified in the NuScale DSRS Scope and Safety Review Matrix), and discuss how the proposed alternative provides an acceptable method of complying with the regulations that underlie the DSRS acceptance criteria. The staff has accepted the content of the DSRS as an alternative method for evaluating whether an application complies with NRC regulations for the NuScale Small Modular Reactor applications, provided that the application does not deviate significantly from the design and siting assumptions made by the NRC staff while preparing the DSRS. If the design or siting assumptions in a NuScale application deviate significantly from the design and siting assumptions the staff used in preparing the DSRS, the staff will use the more general guidance in the SRP, as specified in sections 52.17(a)(1)(xii), 52.47(a)(9), or 52.79(a)(41) of title 10 of the *Code of*

*Federal Regulations*, depending on the type of application. Alternatively, the staff may supplement the DSRS section by adding appropriate criteria to address new design or siting assumptions.

The NRC staff issued a **Federal Register** notice on June 30, 2015 (80 FR 37312), to request public comments on the draft NuScale DSRS Scope and Safety review Matrix (ADAMS Accession No. ML15156B063) and the individual NuScale-specific DSRS sections referenced in the table included in the FRN. A correction **Federal Register** notice was published on July 9, 2015 (80 FR 39454), to identify an additional draft DSRS section for which comments were requested. In response, the NRC received comments from: NuScale Power, LLC, by letter dated August 31, 2015 (ADAMS Accession No. ML15258A081), the Nuclear Energy Institute (NEI) by letter dated August 31, 2015 (ADAMS Accession No. ML15257A012), Mark Thomson by electronic submission dated August 31, 2015 (ADAMS Accession No. ML15292A309), an anonymous submitter by electronic submission dated August 31, 2015 (ADAMS Accession No. ML15292A310), an anonymous submitter by electronic submission dated August 31, 2015 (ADAMS Accession No. ML15292A311), Clinton Ferrara by electronic submission dated August 31, 2015 (ADAMS Accession No. ML15292A333), and Paula Ferrara by electronic submission dated August 31, 2015, (ADAMS Accession No. ML15292A334). Several of these comments have been previously discussed during public meetings held in support of developing the draft DSRS sections. These comments and resolutions have been documented in the DSRS Public Comment Resolution Matrices and are publicly available (ADAMS Package Accession No. ML16083A615).

In the June 30, 2015 **Federal Register** notice, the NRC requested public comments on 115 DSRS sections. The NRC staff determined whether to develop a DSRS section after considering whether significant differences in the functions, characteristics, or attributes of the NuScale design required major revision of the related SRP section guidance, or whether structures, systems, and components identified in the NuScale design are unique and not addressed by the current SRP. Following publication of the draft version of the DSRS sections, the NRC staff revisited these criteria and determined, based on the most recent NuScale design, that it is appropriate to use the related SRP section in lieu of a draft DSRS section

in a number of cases. In these cases the draft DSRS sections have not been issued as final, and the related SRP sections will be used for the NuScale review. In deciding to use the related SRP sections, the staff has not necessarily determined that the SRP sections are wholly applicable without modification. For example, as the NRC staff gains greater understanding of the NuScale design or if the design changes during the review, the staff would assess whether different or supplemental review criteria are needed. Stakeholders who believe that different or supplemental review criteria are needed may provide these views to the NRC staff for consideration during the application review period.

The results of determinations to use the related SRP sections rather than draft DSRS sections, along with other identified issues with the draft NuScale DSRS Scope and Safety Review Matrix, are documented in a separate “transitional” NuScale DSRS Scope and Safety Review Matrix (ADAMS Accession No. ML16076A048). The “transitional” Matrix shows the differences between the draft and final NuScale DSRS Scope and Safety Review Matrices and describes the reasons for these differences. The resulting final list of DSRS titles with corresponding section numbers and ADAMS references are provided in the table below and in ADAMS Package Accession No. ML15355A295.

In the future, should additional SRP sections be developed, the staff will determine at that time their applicability to the NuScale design. In addition, the NRC disseminates information regarding current safety issues and proposed solutions through various means, such as generic communications and the process for treating generic safety issues. When current issues are resolved, the staff will determine the need, extent, and nature of revision that should be made to the SRP and/or DSRS to reflect the new NRC guidance.

### III. Availability of Documents

| Section          | Design-specific review standard title                                                   | ADAMS accession No. |
|------------------|-----------------------------------------------------------------------------------------|---------------------|
| Matrix .....     | NuScale DSRS Scope and Safety Review Matrix (Transitional) .....                        | ML16076A048         |
| Matrix .....     | NuScale DSRS Scope and Safety Review Matrix (Final) .....                               | ML16263A000         |
| 3.5.1.3 .....    | Turbine Missiles .....                                                                  | ML15355A364         |
| 3.7.1 .....      | Seismic Design Parameters .....                                                         | ML15355A384         |
| 3.7.2 .....      | Seismic System Analysis .....                                                           | ML15355A389         |
| 3.7.3 .....      | Seismic Subsystem Analysis .....                                                        | ML15355A402         |
| 3.8.2 .....      | Steel Containment .....                                                                 | ML15355A411         |
| 3.8.4 .....      | Other Seismic Category I Structures .....                                               | ML15355A444         |
| 3.8.5 .....      | Foundations .....                                                                       | ML15355A451         |
| 3.11 .....       | Environmental Qualification of Mechanical and Electrical Equipment .....                | ML15355A455         |
| 4.4 .....        | Thermal and Hydraulic Design .....                                                      | ML15355A468         |
| 5.2.4 .....      | Reactor Coolant Pressure Boundary Inservice Inspection and Testing .....                | ML15355A479         |
| 5.2.5 .....      | Reactor Coolant Pressure Boundary Leakage Detection .....                               | ML15355A505         |
| 5.3.1 .....      | Reactor Vessel Materials .....                                                          | ML15355A513         |
| 5.3.2 .....      | Pressure-Temperature Limits, Upper-Shelf Energy, and Pressurized Thermal Shock .....    | ML15355A526         |
| 5.3.3 .....      | Reactor Vessel Integrity .....                                                          | ML15355A530         |
| 5.4.2.1 .....    | Steam Generator Materials .....                                                         | ML15355A532         |
| 5.4.2.2 .....    | Steam Generator Program .....                                                           | ML15355A535         |
| 5.4.7 .....      | Decay Heat Removal (DHR) System .....                                                   | ML15355A536         |
| BTP 5–4 .....    | Design Requirements of the Decay Heat Removal System .....                              | ML15355A313         |
| 6.2.1 .....      | Containment Functional Design .....                                                     | ML15356A259         |
| 6.2.1.1.A .....  | Containments .....                                                                      | ML15355A544         |
| 6.2.1.3 .....    | Mass and Energy Release Analysis for Postulated Loss-of-Coolant Accidents (LOCAs) ..... | ML15357A327         |
| 6.2.1.4 .....    | Mass and Energy Release Analysis for Postulated Secondary System Pipe Ruptures .....    | ML15356A241         |
| 6.2.2 .....      | Containment Heat Removal Systems .....                                                  | ML15356A267         |
| 6.2.4 .....      | Containment Isolation System .....                                                      | ML15356A332         |
| 6.2.5 .....      | Combustible Gas Control in Containment .....                                            | ML15356A356         |
| 6.2.6 .....      | Containment Leakage Testing .....                                                       | ML15356A388         |
| 6.3 .....        | Emergency Core Cooling System .....                                                     | ML15356A393         |
| 6.6 .....        | Inservice Inspection and Testing of Class 2 and 3 Components .....                      | ML15356A396         |
| 7.0 .....        | Instrumentation and Controls—Introduction and Overview of Review Process .....          | ML15356A416         |
| 7.1 .....        | Instrumentation and Controls—Fundamental Design Principles .....                        | ML15363A293         |
| 7.2 .....        | Instrumentation and Controls—System Characteristics .....                               | ML15363A347         |
| 7.0, App A ..... | Instrumentation and Controls—Hazard Analysis .....                                      | ML15355A316         |
| 7.0, App B ..... | Instrumentation and Controls—System Architecture .....                                  | ML15355A318         |
| 7.0, App C ..... | Instrumentation and Controls—Simplicity .....                                           | ML15355A319         |
| 7.0, App D ..... | Instrumentation and Controls—References .....                                           | ML15355A320         |
| 8.1 .....        | Electric Power—Introduction .....                                                       | ML15356A473         |
| 8.2 .....        | Offsite Power System .....                                                              | ML15356A516         |
| 8.3.1 .....      | AC Power Systems (Onsite) .....                                                         | ML15356A533         |
| 8.3.2 .....      | DC Power Systems (Onsite) .....                                                         | ML15356A552         |
| 8.4 .....        | Station Blackout .....                                                                  | ML15356A570         |
| 9.1.2 .....      | New and Spent Fuel Storage .....                                                        | ML15356A584         |
| 9.1.3 .....      | Spent Fuel Pool Cooling and Cleanup System .....                                        | ML15356A595         |
| 9.3.4 .....      | Chemical and Volume Control System .....                                                | ML15356A622         |
| 9.3.6 .....      | Containment Evacuation and Flooding Systems .....                                       | ML15356A637         |
| 9.5.2 .....      | Communications Systems .....                                                            | ML15363A400         |
| 10.2.3 .....     | Turbine Rotor Integrity .....                                                           | ML15356A700         |
| 10.3 .....       | Main Steam Supply System .....                                                          | ML15355A322         |
| 10.4.7 .....     | Condensate and Feedwater System .....                                                   | ML15355A331         |
| 11.1 .....       | Source Terms .....                                                                      | ML15355A333         |

| Section           | Design-specific review standard title                                                                                                                                                   | ADAMS accession No. |
|-------------------|-----------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------|---------------------|
| 11.2 .....        | Liquid Waste Management System .....                                                                                                                                                    | ML15355A334         |
| 11.3 .....        | Gaseous Waste Management System .....                                                                                                                                                   | ML15355A335         |
| 11.4 .....        | Solid Waste Management System .....                                                                                                                                                     | ML15355A336         |
| 11.5 .....        | Process and Effluent Radiological Monitoring Instrumentation and Sampling Systems .....                                                                                                 | ML15355A337         |
| 11.6 .....        | Guidance on Instrumentation and Control Design Features for Process and Effluent Radiological Monitoring, and Area Radiation and Airborne Radioactivity Monitoring.                     | ML15355A338         |
| 12.2 .....        | Radiation Sources .....                                                                                                                                                                 | ML15350A320         |
| 12.3–12.4 .....   | Radiation Protection Design Features .....                                                                                                                                              | ML15350A339         |
| 12.5 .....        | Operational Radiation Protection Program .....                                                                                                                                          | ML15350A341         |
| 14.2 .....        | Initial Plant Test Program—Design Certification and New License Applicants .....                                                                                                        | ML15355A339         |
| 14.3.5 .....      | Instrumentation and Controls—Inspections, Tests, Analyses, and Acceptance Criteria .....                                                                                                | ML15355A340         |
| 15.0 .....        | Introduction—Transient and Accident Analyses .....                                                                                                                                      | ML15355A302         |
| 15.0.3 .....      | Design Basis Accidents Radiological Consequence Analyses for NuScale SMR Design .....                                                                                                   | ML15355A341         |
| 15.1.1–15.1.4 ..  | Decrease in FW Temperature, Increase in FW Flow, Increase in Steam Flow and Inadvertent Opening of the Turbine Bypass System or Inadvertent Operation of the Decay Heat Removal System. | ML15355A303         |
| 15.1.5 .....      | Steam System Piping Failures Inside and Outside of Containment .....                                                                                                                    | ML15355A304         |
| 15.1.6 .....      | Loss of Containment Vacuum .....                                                                                                                                                        | ML15355A305         |
| 15.2.1–15.2.5 ... | Loss of External Load; Turbine Trip; Loss of Condenser Vacuum; Closure of Main Steam Isolation Valve (BWR); and Steam Pressure Regulator Failure (Closed).                              | ML15355A306         |
| 15.2.6 .....      | Loss of Non-Emergency AC Power to the Station Auxiliaries .....                                                                                                                         | ML15363A348         |
| 15.2.7 .....      | Loss of Normal Feedwater Flow .....                                                                                                                                                     | ML15355A307         |
| 15.2.8 .....      | Feedwater System Pipe Breaks Inside and Outside Containment .....                                                                                                                       | ML15355A308         |
| 15.5.1–15.5.2 ... | Chemical and Volume Control System Malfunction that Increases Reactor Coolant Inventory .....                                                                                           | ML15363A397         |
| 15.6.5 .....      | LOCAs Resulting From Spectrum of Postulated Piping Breaks Within the Reactor Coolant Pressure Boundary.                                                                                 | ML15355A309         |
| 15.6.6 .....      | Inadvertent Opening of the Emergency Core Cooling System .....                                                                                                                          | ML15355A310         |
| 15.9A .....       | Thermal-hydraulic Stability .....                                                                                                                                                       | ML15355A311         |
| 16.0 .....        | Technical Specifications .....                                                                                                                                                          | ML15355A312         |

Dated at Rockville, Maryland, this 21st day of October 2016.

For the Nuclear Regulatory Commission.

**Frank Akstulewicz,**

*Director, Division of New Reactor Licensing,  
Office of New Reactors.*

[FR Doc. 2016-26210 Filed 10-28-16; 8:45 am]

**BILLING CODE 7590-01-P**

## NUCLEAR REGULATORY COMMISSION

[Docket Nos.: 52-029 and 52-030; NRC-2008-0558]

### Duke Energy Florida, LLC; Levy Nuclear Plant Units 1 and 2

**AGENCY:** Nuclear Regulatory Commission

**ACTION:** Notice of intent to enter into a modified indemnity agreement.

**SUMMARY:** The U.S. Nuclear Regulatory Commission (NRC) is issuing a notice of intent to enter into a modified indemnity agreement with Duke Energy Florida, LLC, (DEF) to operate Levy Nuclear Plant Units 1 and 2 (LNP 1 and 2). The NRC is required to publish notice of its intent to enter into an indemnity agreement which contains provisions different from the general form found in the NRC's regulations. A modification to the general form is necessary to accommodate the unique timing provisions of a combined license (COL).

**DATES:** On October 20, 2016, the Commission authorized the Director of the Office of New Reactors to issue COLs to DEF to construct and operate LNP 1 and 2. The modified indemnity agreement would be effective upon issuance of the COLs.

**ADDRESSES:** Please refer to Docket ID NRC-2008-0558 when contacting the NRC about the availability of information regarding this document. You may obtain publicly-available information related to this document using any of the following methods:

- *Federal Rulemaking Web site:* Go to <http://www.regulations.gov> and search for Docket ID NRC-2008-0558. Address questions about NRC dockets to Carol Gallagher; telephone: 301-415-3463; email: [Carol.Gallagher@nrc.gov](mailto:Carol.Gallagher@nrc.gov). For technical questions, contact the individual listed in the **FOR FURTHER INFORMATION CONTACT** section of this document.

- *NRC's Agencywide Documents Access and Management System (ADAMS):* You may obtain publicly-available documents online in the ADAMS Public Documents collection at <http://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "ADAMS Public Documents" and then select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, 301-415-4737, or by email to [pdr.resource@nrc.gov](mailto:pdr.resource@nrc.gov).

- *NRC's PDR:* You may examine and purchase copies of public documents at the NRC's PDR, Room O1-F21, One White Flint North, 11555 Rockville Pike, Rockville, Maryland 20852.

#### FOR FURTHER INFORMATION CONTACT:

Donald Habib, Office of New Reactors, U.S. Nuclear Regulatory Commission, Washington DC 20555-0001; telephone: 301-415-1035, email: [Donald.Habib@nrc.gov](mailto:Donald.Habib@nrc.gov).

#### SUPPLEMENTARY INFORMATION:

##### I. Background

On October 20, 2016, the Commission authorized issuance of COLs to DEF for LNP 1 and 2. These COLs would include a license pursuant to part 70 of title 10 of the *Code of Federal Regulations* (10 CFR), "Domestic Licensing of Special Nuclear Material." Pursuant to 10 CFR 140.20(a)(1)(iii), the NRC will execute and issue agreements of indemnity effective on the date of a license under 10 CFR part 70 authorizing the licensee to possess and store special nuclear material at the site of the nuclear reactor for use as fuel in operation of the nuclear reactor after issuance of an operating license for the reactor. The general form of indemnity agreement to be entered into by the NRC with DEF is contained in 10 CFR 140.92, "Appendix B—Form of Indemnity Agreement with licensees furnishing insurance policies as proof of financial protection."